Candu Reactor Severe Accident Analysis For Accident Management

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Candu Reactor Severe Accident Analysis

CANDU SEVERE ACCIDENT ANALYSIS - pub.ro
The CANDU reactor has moderator calandria vessel as ultimate heat sink during severe accident development, which acts also as a core debris catcher 2 CANDU severe accidents analysis Nuclear reactor severe accidents are important in terms of consequences: radioactive …
Severe Core Damage Accident Progression within a CANDU ...
The progression of a severe core damage accident in a CANDU reactor is analyzed using the MAAP4 CANDU code, which is the CANDU-version of the MAAP code AECL and Ontario Power Generation Inc, in co-operation with Fauske and Associates Inc, have developed the MAAP4 CANDU code for severe core damage accident analysis in a CANDU reactor
Severe Core Damage Accidents & MAAP4 CANDU.
Pg 3 Introduction • Presentation addresses Severe Core Damage Accident Analysis using MAAP4 CANDU • Severe Core Damage Accident – Accident in which substantial damage is done to the reactor core structure whether or not there are serious off-site consequences – Reactor Cooling System and Moderator back-up heat sinks are unavailable In ACR-700, RWS must also fail (very unlikely …
CANDU Severe Core Damage Experiments and Analysis
• The progression of a Severe Core Damage Accident in a CANDU reactor is analysed by the MAAP-CANDU code • MAAP (Modular Accident Analysis Program) is an integrated code designed for Severe Accident Consequence Analysis in nuclear plants • MAAP is owned by EPRI • MAAP developed by Fauske & Associates Inc (FAI), used by 40
3D NUMERICAL ANALYSIS OF MOLTEN DEBRIS DURING LATE ...
pool formation and movement in case of a severe accident at a CANDU 6 reactor Even if a severe accident never occurred in a CANDU reactor, the

accident in Fukushima Daiichi, Japan illustrates why the research needs to be made to be able to understand the consequences of such an event During a postulated severe accident in a CANDU power plant

THERMAL-HYDRAULIC ASPECTS OF PROGRESSION TO ...

REACTOR SAFETY KEYWORDS:severe accidents, CANDU reactors, thermal hydraulics THERMAL-HYDRAULIC ASPECTS OF PROGRESSION TO SEVERE ACCIDENTS IN CANDU REACTORS JOHN C LUXAT* McMaster University 1280

Chapter 7 - Accident Analysis - nuceng.ca

Chapter 7 - Accident Analysiswpd Rev 8 November 9, 2009 22:26:44 vgs Chapter 7 - Accident Analysis Introduction This Chapter provides more specific information on performing accident analysis for CANDU The process by which initiating events are selected is first discussed - some of this will review what we covered in Chapter 1

Severe Accident Progression Without Operator Action

The results of the MAAP4-CANDU (computer code) analysis that was produced to support the Darlington Level 2 PSA were used to verify the accident progression of a severe accident without operator actions The specific accident described here is a prolonged station blackout scenario, where

Analysis of Severe Accidents in Pressurized Heavy Water ...

Accident Analysis This publication provides a description of factors important to severe accident analysis, an overview of severe accident phenomena and the current status in their modelling, categorization of available computer codes, and differences in approaches for various applications of severe accident analysis

CANDU Safety #10: Design and Analysis Process

24-May-01 CANDU Safety - #10 - Design and Analysis Processppt Rev 0 3 Safety Criteria λ The Canadian Atomic Energy Control Board (AECB) is the regulatory body for nuclear power plants in Canada λ The AECB defines the – basic safety criteria for normal operation and accident conditions

CANDU Safety #16: Large Loss-of-Coolant Accident with ...

24-May-01 CANDU Safety - #16 - LOCA loeccppt Rev 0 2 Overview λ Event sequence for a large break loss-of-coolant accident with loss of emergency core cooling λ Acceptance criteria used to assess the results of the analysis λ Fuel and pressure tube behaviour during transient λ …

RELAP/SCDAP SIMULATION RESULTS FOR CANDU 6 ...

code ([1], [2], [3],[13]) was used for this severe accident analysis 2 CANDU-6 plant models CANDU-6 reactor is a Canadian designed Pressurized Heavy Water Reactor (PHWR) type, having a gross capacity of about 700 MW(e), using heavy water both for moderator and for primary coolant, in separate circuits, and natural uranium for fuel

Analysis Report, Generic CANDU Probabilistic Safety ...

4 to develop databases for reference CANDU 6 and CANDU 9 plants, in order to perform severe core damage progression analysis This report also describes the steps that are involved in performing a Level II PSA, for which the Modular Accident Analysis Program CANDU …

Enhanced CANDU 6 - SNC-Lavalin

pressure tube reactor Heavy water (D 2 O) is a natural form of water that is used as a moderator to slow down the neutrons in the reactor, enabling the use of natural uranium as fuel This feature is unique to CANDU reactors The choice of D 2 O as the moderator also allows other fuel cycles to be used in CANDU reactors

BEPU ANALYSIS OF A STATION BLACKOUT IN CANDU ...

Severe accident, CANDU, RELAP/SCDAPSIM, Best estimate, Uncertainty 1 INTRODUCTION One of the general attributes of a methodology to perform accident analysis of a nuclear power plant for the safety assessment is directly connected with the availability of qualified tools and analytical procedures suitable for this purpose

Collapse Criterion for the CANDU 6 Calandria Vault Floor ...

Keywords: CANDU, calandria vault, molten core–concrete interaction, failure, finite-element analysis 1 Introduction During a postulated severe accident in a generic CANDU 6 nuclear reactor, decay heat could cause the fuel channels to heat up and collapse to the bottom of the calandria vessel and then melt into a mixture called corium

MAAP4 CANDU Analysis of a Generic CANDU-6 Plant

that later code versions can model other CANDU designs, including the Advanced CANDU Reactor (ACR) MAAP4-CANDU is a consequence analysis tool to assist the probabilistic safety assessment of CANDU reactors during severe accidents; the code is not designed for analysing design-basis accidents

Verification by analytical means of the efficiency of some ...

analysis of the behavior of a CANDU-6 NPP in case of a Station Black-Out (SBO) initiating event occurring with reactor at full power and with the nuclear fuel at equilibrium condition In certain accident conditions, when the cooling of the nuclear fuel cannot be ensured, the SBO event can progress to a severe accident with reactor core damage

Application of Technologies in CANDU Reactors to Prevent ...

generated during CCI This has been recognized in the Candu Owners Group (COG) SAMG and appropriate guidance has been provided •MAAP4-CANDU analyses confirmed that containment is steam inerted in many of the severe accident sequences analysed

Candu Reactor Severe Accident Analysis For Accident ...

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